



HEATING TOKAMAK PLASMA WITH FAST IONS

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OUTLINE OF LECTURE 2A

- Ohmic heating of plasmas
- Evolution of fast ion distribution function in energy space, ion and electron heating
- Drift orbits
- Neutral beam injection (NBI)
- Ion Cyclotron Resonance Heating (ICRH)
- Fusion products
- Summary

OHMIC HEATING

- All tokamaks are heated initially by the plasma current
Ohmic power = $I_p V = [I_p]^2 R$
Plasma resistivity $R \sim [T_e]^{-3/2}$
- As the plasma gets hotter:
 - its resistivity gets smaller – the ohmic power falls
 - the energy losses increase - τ_E gets smaller
 - there is a maximum temperature ~ 5 keV that can be reached by Ohmic heating
- **Additional heating techniques** are needed to obtain **10-20 keV** temperature thermal ions. Heating plasma up to this temperature range with a **small density population of “fast” ions** with energy $E_f \gg T_i \sim T_e$ is the most attractive and natural way

HEATING THE PLASMA WITH FAST ION POPULATION

- Fast particle population (e.g. fusion-born **alpha-particles**) has **high energy**,

$$E_f \gg T_i \sim T_e$$

but **low density**,

$$n_f \ll n_e$$

- Energy content of the fast particle population may be comparable to thermal plasma energy content,

$$\beta_f = n_f E_f \sim \beta_{\text{therm}}$$

- **Source** of the fast ions may be **isotropic** in velocity space (fusion-born **alpha-particles**) or **anisotropic** (a **beam** of fast ions)
- The fast ions transfer their energy to thermal ions and electrons by **Coulomb collisions**.

FAST ION EVOLUTION IN MAXWELLIAN PLASMA-1

- Consider evolution of fast ion distribution function, $f(v_x, v_y, v_z)$, due to the **Coulomb collisions with thermal plasma species** in homogeneous plasma. An axial symmetry of the distribution function is assumed during the evolution:

$$f(v_x, v_y, v_z) = f(v, \vartheta); \quad v = \sqrt{v_{\parallel}^2 + v_{\perp}^2}; \quad \vartheta = \tan^{-1}(v_{\perp} / v_{\parallel}); \quad v_{\perp} = \sqrt{v_x^2 + v_y^2}; \quad v_{\parallel} = v_z$$

- This evolution can be described by **Fokker-Planck equation** for fast ion distribution function $f(v, \vartheta)$ with initial velocity V_0
- Typical range of the fast ion velocities satisfies $v_i \ll V_0 \ll v_e$. **Neglecting self-collisions between fast ions**, Fokker-Planck equation becomes linear and it has the form

FAST ION EVOLUTION IN MAXWELLIAN PLASMA-2

$$\frac{\partial f}{\partial t} = \frac{V_0^3}{\tau v^2} \left\{ \frac{\partial}{\partial v} \left[\frac{V_0^2 a(v)}{2v} \frac{\partial f}{\partial v} + b(v) f \right] + \frac{c(v)}{V_0} \cdot \frac{1}{\sin \mathcal{G}} \frac{\partial}{\partial \mathcal{G}} \left[\sin \mathcal{G} \frac{\partial f}{\partial \mathcal{G}} \right] \right\} - \nu f + pF$$

where

$$a(v) = \frac{T_e}{E_0} \left\{ \tilde{Z}_2 + \frac{4}{3\sqrt{\pi}} \frac{m_0}{m_e} \left(\frac{v}{v_e} \right)^3 \right\}; \quad b(v) = \tilde{Z}_1 + \frac{4}{3\sqrt{\pi}} \frac{m_0}{m_e} \left(\frac{v}{v_e} \right)^3; \quad c(v) = Z_{eff} \frac{V_0}{2v} + \frac{2}{3\sqrt{\pi}} \frac{V_0}{v_e} = Z_{eff} \frac{V_0}{2v} \left(1 + \frac{4}{3\sqrt{\pi}} \frac{v}{v_e} \right);$$

$$\tau = \frac{1}{\pi\sqrt{2}} \frac{E_0^{3/2} m_0^{1/2}}{Z_b^2 e^4 n_e L}$$

$$\tilde{Z}_1 = \frac{m_0}{n_e} \sum_i \frac{Z_i^2 n_i}{m_i}; \quad \tilde{Z}_2 = \frac{m_0}{n_e T_e} \sum_i \frac{Z_i^2 n_i T_i}{m_i}; \quad Z_{eff} = (1/n_e) \sum_i Z_i^2 n_i;$$

L is the Coulomb logarithm, m_0 the fast ion mass, E_0 , V_0 are the initial energy and velocity of fast ions and **two last terms** in the right-hand-side of this equation

represent **sink and source of the energetic ions**. Source: $2\pi \int_0^\infty v^2 dv \int_0^\pi F(v - v_0; \mathcal{G}) d\mathcal{G} = 1$

VELOCITY SPACE DIFFUSION AND DRAG

$$\frac{\partial f}{\partial t} = \frac{V_0^3}{\tau v^2} \left\{ \frac{\partial}{\partial v} \left[\frac{V_0^2 a(v)}{2v} \frac{\partial f}{\partial v} + b(v) f \right] + \frac{c(v)}{V_0} \cdot \frac{1}{\sin \vartheta} \frac{\partial}{\partial \vartheta} \left[\sin \vartheta \frac{\partial f}{\partial \vartheta} \right] \right\} - v f + pF$$

- **Diffusion** of $f(v, \vartheta)$ is given by two terms proportional to $a(v)$ and $c(v)$, with the pitch-angle scattering $c(v)$ usually dominating over the velocity diffusion $a(v)$
- The diffusion gives a **broadening (in velocity space)** of the distribution function
- The **drag (also called “dynamical friction”)** is represented by $b(v)$.
- The drag gives a **deceleration (slowing-down)** of the distribution function

CLASSICAL SCHEME OF PLASMA HEATING BY FAST IONS-1

- The fast ions transfer their energy to thermal ions and electrons by Coulomb collisions. If the **energy** of the fast ions is less than a critical value

$$E_{crit} = 14.8 A_f T_e \left(\sum_i n_i Z_i^2 / n_e A_i \right)^{2/3},$$

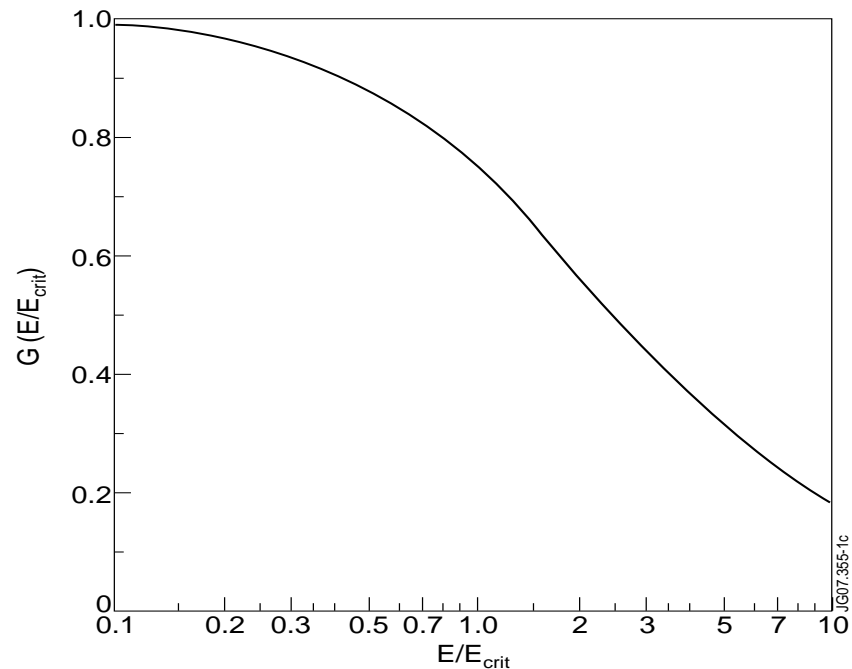
power flows mainly **to thermal ions** rather than to electrons. Here, A_f , A_i are atomic masses of fast ions and thermal ions, T_e is electron temperature, n_i and n_e are ion and electron densities, and Z_i is atomic number of thermal ions.

- **Above** the critical energy, the **drag** dominates in Fokker-Planck equation
- **Below** the critical energy the **diffusion** dominates

CLASSICAL SCHEME OF PLASMA HEATING BY FAST IONS-2

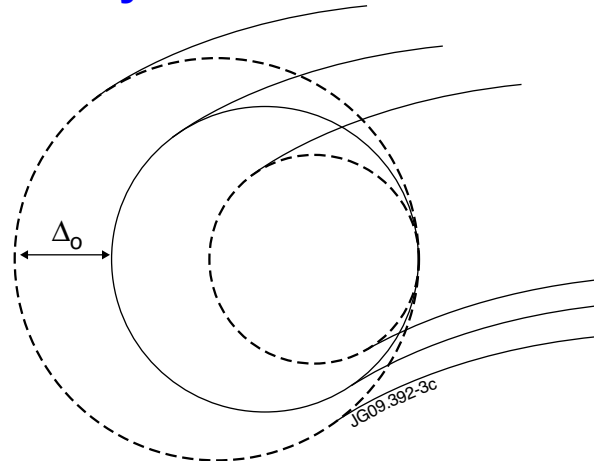
- The amount of energy going from ions with initial energy E into plasma ions is

given by Stix formula $G_i = \frac{E_{crit}}{E} \int_0^{E/E_{crit}} \frac{dy}{1+y^{3/2}}$, and $G_i(E/E_{crit})$ is illustrated below



ION ORBITS IN A TORUS - 1

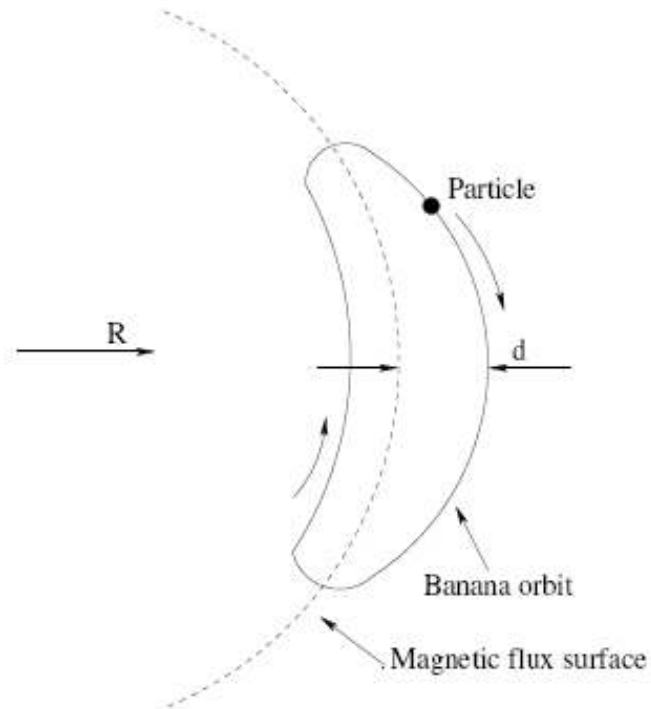
- In a straight magnetic field the ions move along helical orbits centered on a field line. The radius of the orbit is the Larmor radius
- In a torus the ion orbits are centered on **drift surfaces** which are displaced from the magnetic surface by value $\Delta_{\text{Orbit}} \equiv \Delta_0$



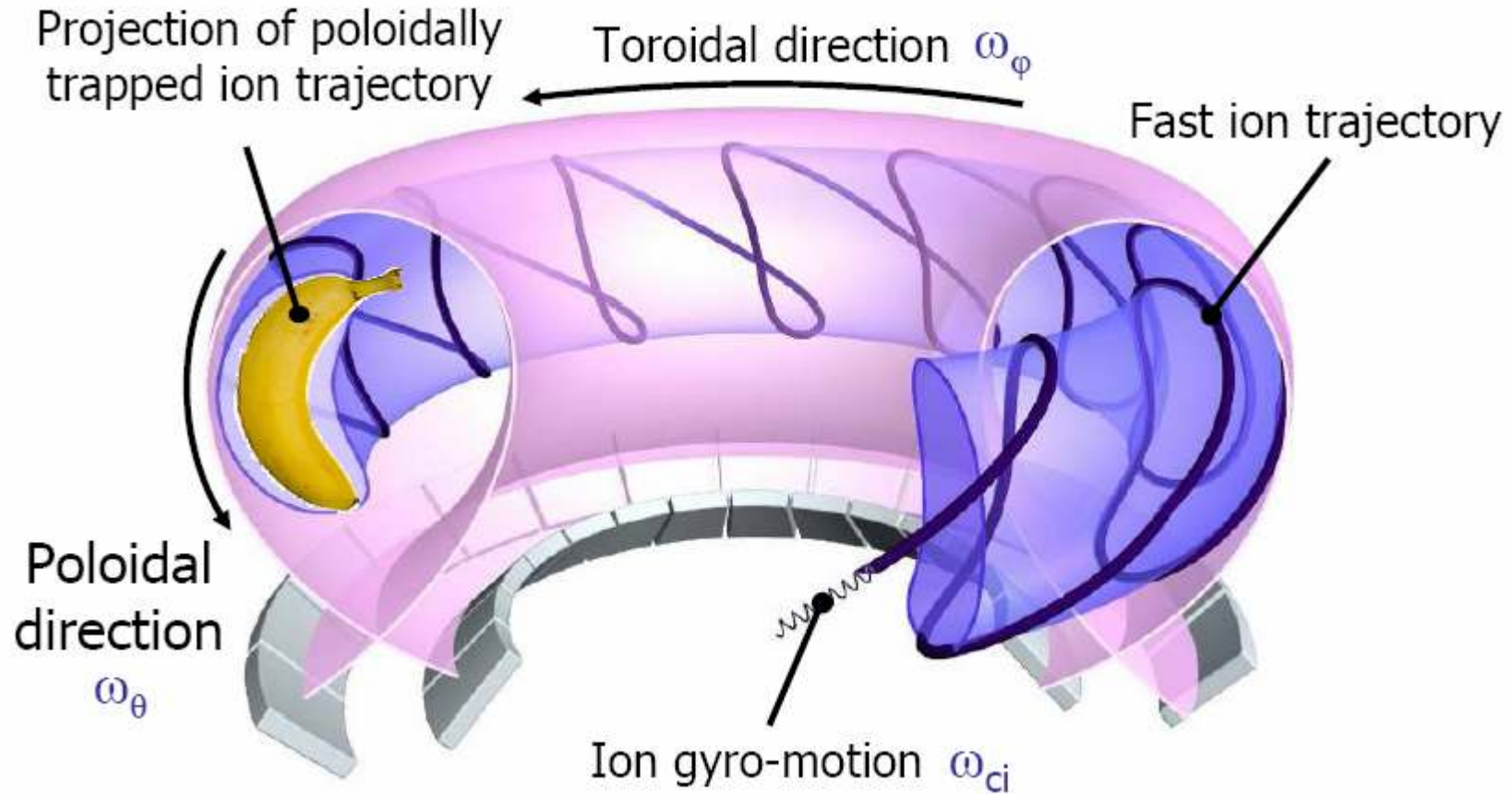
- The Larmor radius depends on B_T . The displacement Δ_0 depends on B_P (and thus on current I_P) and is:
 - Inwards** for ions moving in the same direction as I_P
 - Outwards** for ions moving in the opposite direction

ION ORBITS IN A TORUS - 2

- Some ions are reflected by the strong toroidal field at the inside of the torus
- The projection of the ion orbit onto the poloidal plane looks like a “banana”



FAST PARTICLE ORBITS: TRAPPED ORBITS



NEUTRAL BEAM INJECTION - 1

- Ions from the ion source accelerate by grids to high energy
- Then they pass through the neutraliser and become neutral high energy atoms
- The neutral beam penetrates the tokamak magnetic fields. The **penetration** of the beam depends on the **NBI energy, mass and on the plasma density**
- Within plasma neutrals are ionized by collisions with thermal ions & electrons
- These fast ions are trapped by the tokamak magnetic fields
- NBI systems on JET, JT-60U, TFTR, DIII-D **have $E \leq E_{crit}$** so they heat **IONS**
- NBI systems on MAST & NSTX, (and future NBI on ITER) **have $E > E_{crit}$** so they heat **ELECTRONS**

NEUTRAL BEAM INJECTION - 2

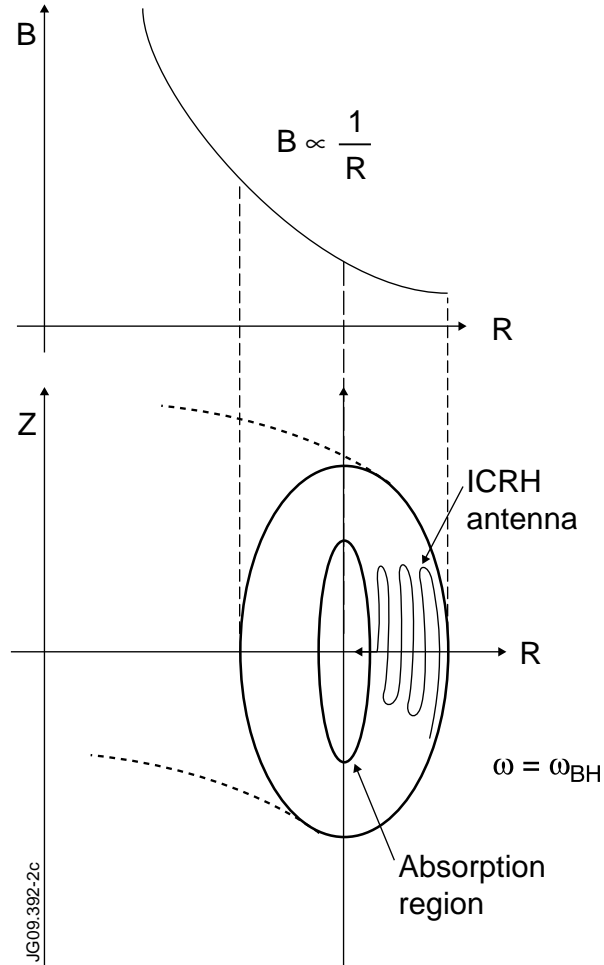
Advantages

- Efficient heating of **ions**
- High power capability (40 MW on TFTR, 24 MW on JET)
- Drives plasma rotation (stabilising lock modes)
- **Fuelling!**
- Some current drive

Disadvantages

- Need MeV energy beams for penetrating in a reactor → Negative ion source for NBI is needed
- Heating not well localised
- Large aperture

ION CYCLOTRON RESONANCE HEATING - 1



ION CYCLOTRON RESONANCE HEATING - 2

Advantages

- Localised heating
- Hydrogen minority ICRH creates H minority with $E > E_{\text{crit}}$ - it heats **ELECTRONS**
- However, heating of IONS is also possible (e.g. ^3He minority in DT plasma)
- Some current drive

Disadvantages

- Antenna inside the vessel
- Low power capability
- Plasma coupling may be a problem in, e.g. H-mode with ELMs

ALPHA PARTICLE HEATING AND BURNING PLASMAS

- Burning plasmas: auxiliary heating used, but significant **plasma self-heating by fusion alphas** exists → plasma becomes **exothermic** medium
- Alphas born at 3.5 MeV have $E > E_{\text{crit}}$ → they mostly heat **ELECTRONS**
- However, electrons do not fuse, we need to heat ions with fusion born alpha-particles. The problem of **re-directing** the alpha-heating from electrons to **IONS** is called “alpha-channeling”

FAST PARTICLES IN JET DT DISCHARGE WITH 16 MW FUSION

	E, keV	E_{crit}/T_e	E/E_{crit} for $T_e=14$ keV	$G_i/G_e = G_i/(1-G_i)$
Fusion alpha-particles	$3.52 \cdot 10^3$	33	7.62	0.3
Deuterium NBI	140	16.5	0.61	5.67
Tritium NBI	160	25	0.46	9
ICRH-accelerated hydrogen	≈ 500	8.25	4.33	0.54

Main types of energetic ions in JET D-T plasma (D:T=50:50, JET pulse #42976).

FAST PARTICLES IN JET

Machine	JET	JET	JET	JET	ITER
Type of fast ions	Hydrogen	He ³	He ⁴	Alpha	Alpha
Source	ICRH tail	ICRH tail	ICRH tail	Fusion	Fusion
Mechanism	minority	minority	3 rd harmonic of NBI	DT nuclear	DT nuclear
τ_S (s)	1.0	0.9	0.4	1.0	0.8
$P_f(0)$ (MW/m ³)	0.8	1.0	0.5	0.12	0.55
$n_f(0) / n_e(0)$ (%)	1.0	1.5	1.5	0.44	0.85
$\beta_f(0)$ (%)	2	2	3	0.7	1.2
$\langle \beta_f \rangle$ (%)	0.25	0.3	0.3	0.12	0.3
$\max R\nabla\beta_f $ (%)	≈ 5	≈ 5	5	3.5	3.8

Slowing down time: τ_S ; heating power per volume at the magnetic axis: $P_f(0)$; ratio of the on-axis fast ion density to electron density: $n_f(0) / n_e(0)$; on-axis fast ion beta: $\beta_f(0)$; volume-averaged fast ion beta: $\langle \beta_f \rangle$ (%); normalised radial gradient of fast ion beta, $\max |R\nabla\beta_f|$. Predicted values of similar parameters are also given for alpha-particles in ITER.

SUMMARY

- Ohmic heating has maximum 5 keV temperature; heating by small population of fast ions is required to obtain 10-20 keV optimum for DT
- There is a critical energy E_{crit} at which the heating flows from fast ions to electrons and ions become equal. Fast ions with $E > E_{\text{crit}}$ heat mostly electrons, $E < E_{\text{crit}}$ - heat ions.
- Two dominant effects determine evolution of fast ions in velocity space: drag and diffusion
- Drift orbits of fast ions may be passing or trapped
- NBI and ICRH are two main techniques generating various types of fast ions
- **Alpha particles** mostly heat electrons